

# Used Fuel Disposition Campaign

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## *Fuel Cycle Technologies Technical Basis for Disposal of Spent Nuclear Fuel and High-Level Radioactive Waste in Salt*

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**Nuclear Waste Technical Review Board (NWTRB) Meeting**  
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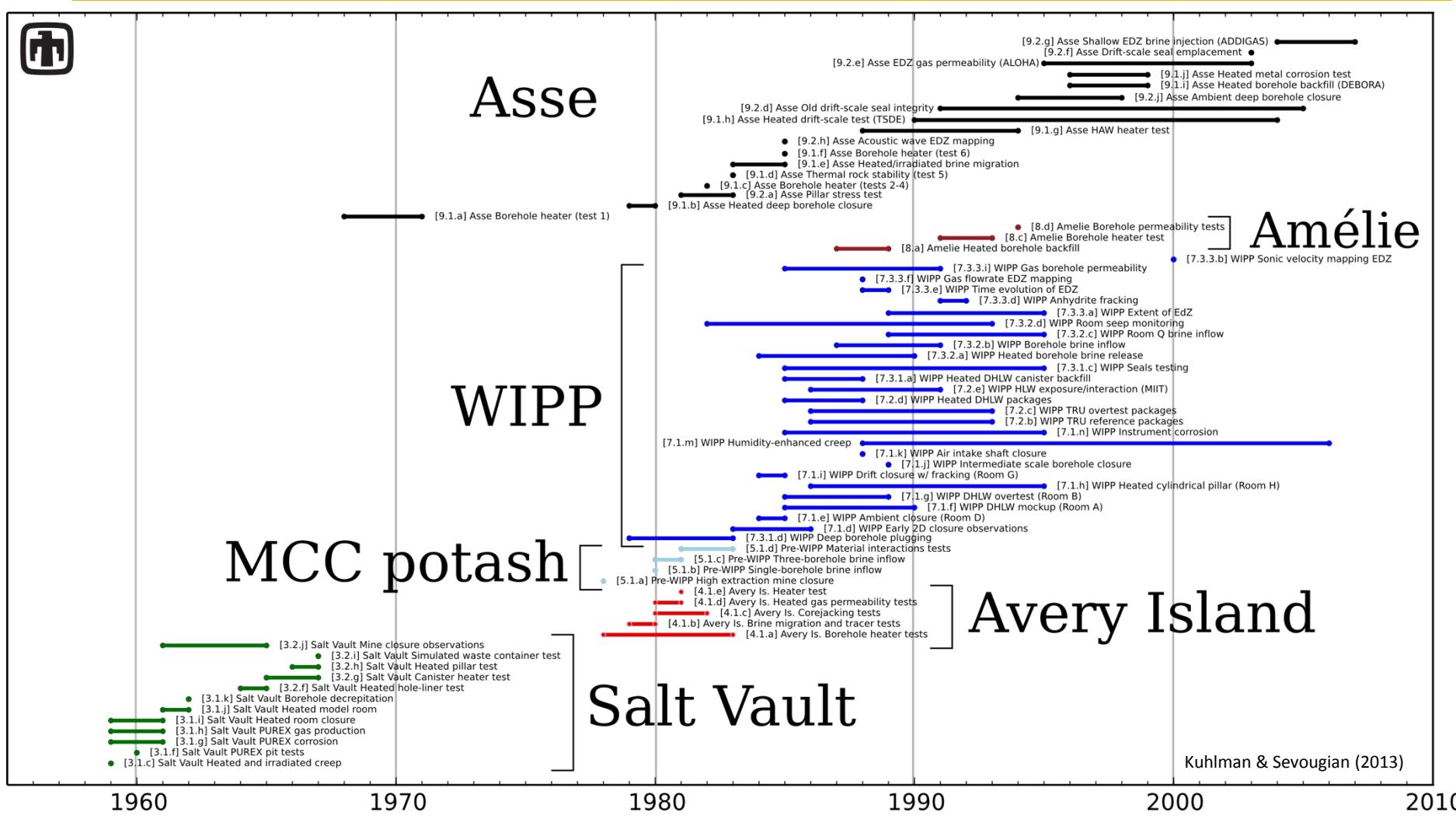
■ ***What is a “Technical Basis”?***

- *Achieved through iterative process:*
  1. Understand Relevant Processes
  2. Develop Conceptual/Mathematical/Numerical Models
  3. *Parameterize/Validate Models with Observations* 
  4. Quantify Limitations and Uncertainty in Models

■ **Outline:**

- *Highlights of HLW-related testing in salt*
- *What has been learned?*
- *What remains?*

## ***Heated Salt In Situ Testing Timeline***



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## U Texas Lab Testing

### ■ NAS panel

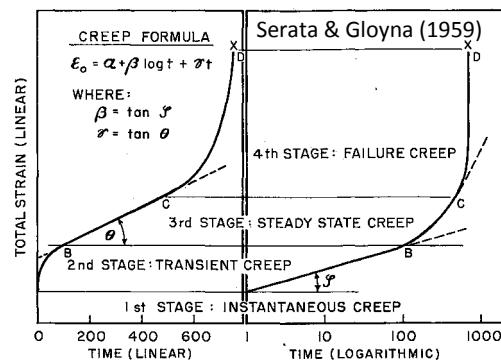
- disposal of liquid reprocessing waste in salt domes (Hess et al., 1957)

### ■ U Texas Austin performed early lab testing

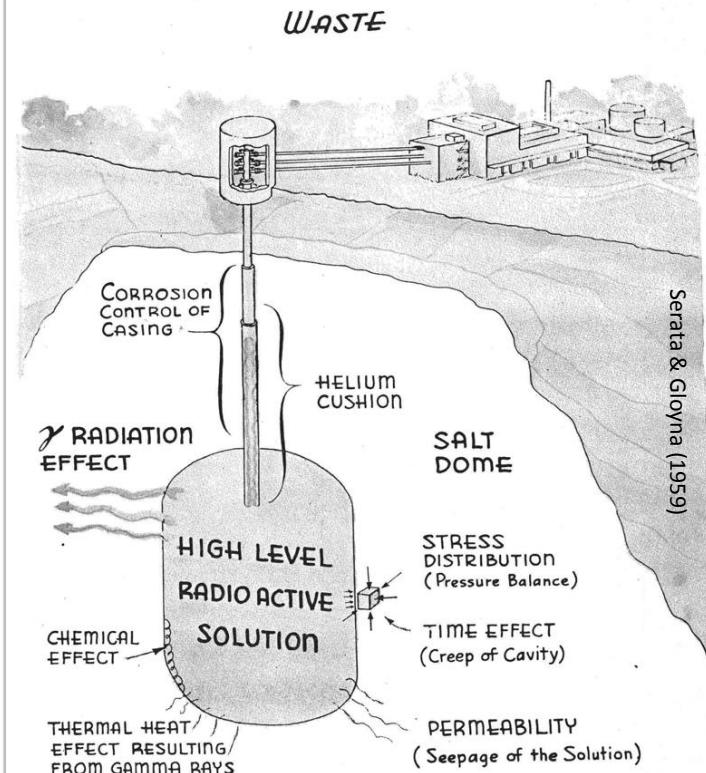
- Uniaxial creep ( $\leq 410^\circ \text{C}$ )
- Cavity closure
- Salt permeability ( $k$ ) testing
  - *He, brine, and kerosene flow*
  - *Crystals are impermeable*
- Closure observations Grand Saline Mine (Dallas, TX)

### ■ Learned:

- *Early geomechanical tests validated thermo-mechanical theory*



## REACTOR FUEL STORAGE PROBLEM RESEARCH CONCEPTS FOR STORING



# Used Fuel Disposition

## ORNL Pre Salt Vault: Hutchinson, KS

### ■ **Heated liquid PUREX waste in salt**

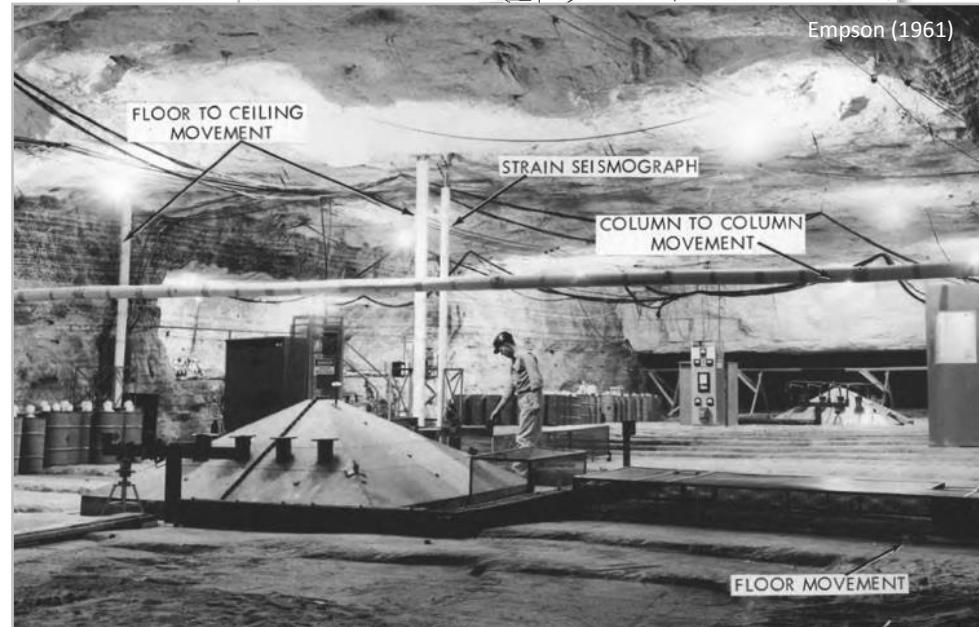
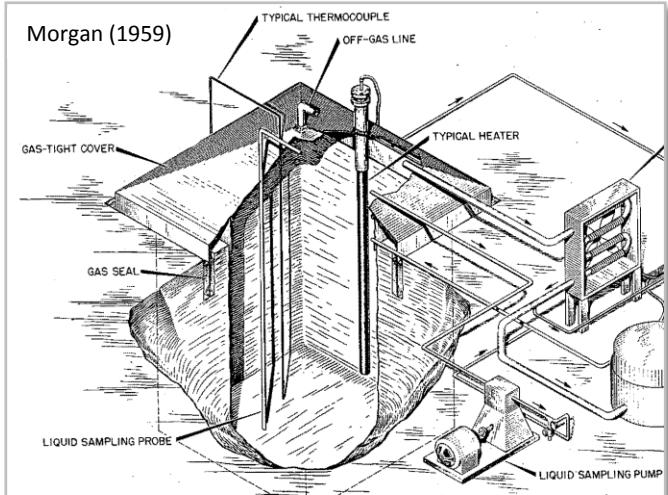
- Lab test in 2' salt blocks (1/10 scale) for 2 months (1959)
- Field test in pits in mine floor (1/5 scale) for 2.5 months (1960)
- Full-scale test in mine floor for 13 months (1961)

### ■ **Monitored waste/salt behavior**

- Room + cavity creep closure
- Solids precipitation/deposition
- Corrosion of materials
- Gas generation

### ■ **Learned:**

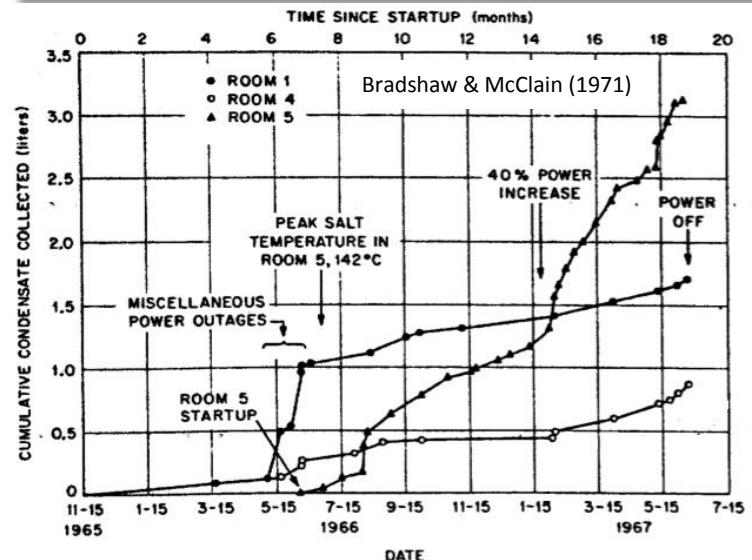
- *Direct liquid disposal infeasible due to gas generation & cavity stability*



# Used Fuel Disposition

## ORNL Project Salt Vault: Lyons, KS

- **Lyons AEC solid waste demo**
- **Hot borehole (July '62)**
  - Two 5-kW heaters
  - Salt to  $>350^\circ \text{C}$  (major decrepitation)
- **3 heater test sites ('65 – '67)**
  - 7 boreholes per site (10.5 kW)
  - Change out radioactive sources
- **Heated Pillar Creep ('66 – '67)**
  - Driven by 22 heaters (33 kW)
- **Learned:**
  - Significant brine from non-salt layers
  - Decrepitation can be issue
  - Brine inclusion migration to heaters
  - Without numerical simulations



# Used Fuel Disposition

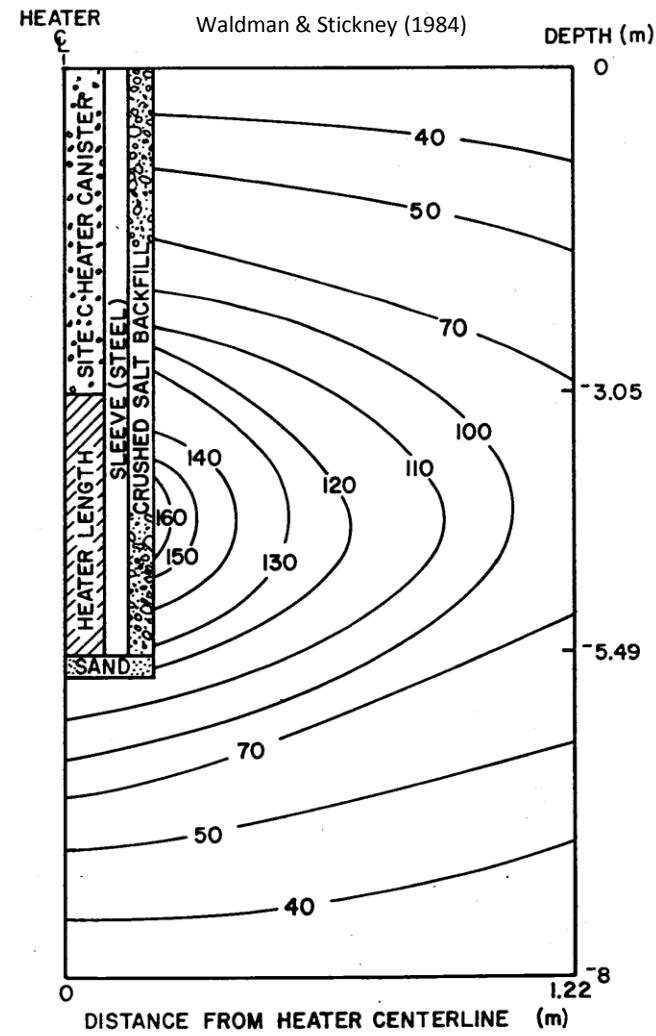
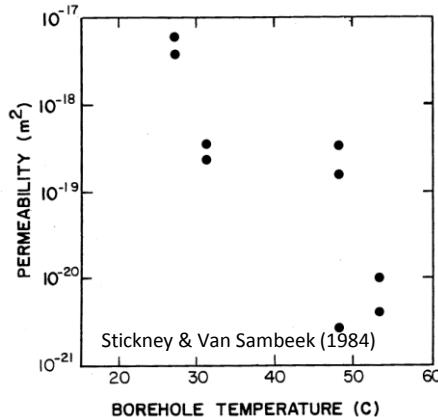
## OWI/ONWI (RESPEC) Avery Island, LA

### ■ Site C heater test ('78 – '83)

- Central + 8 guard heaters (5.6 kW)
- Heater power constant for 5 years
- Salt  $k$  testing using gas flow
- Thermal conductivity ( $\alpha$ ) salt/backfill

### ■ Learned:

- Salt  $k \approx 10^4$  decrease with heating (healing DRZ) due to creep + thermal expansion



# Used Fuel Disposition

## OWI/ONWI (RESPEC) Avery Island, LA

### ■ Brine migration test ('79 – '80)

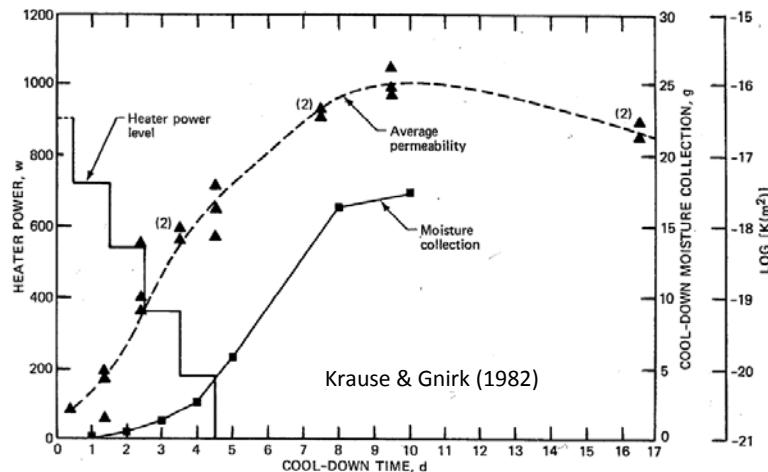
- Unheated/heated boreholes
- Tracer test (Deuterium)

### ■ Gas permeability tests

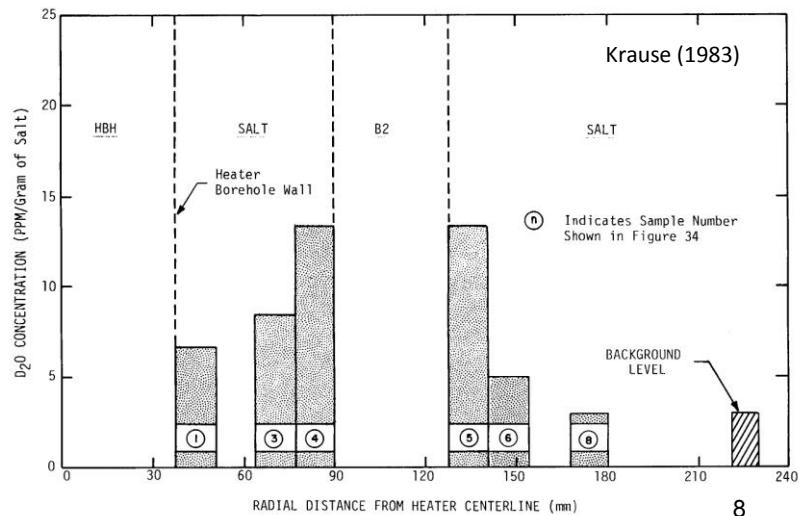
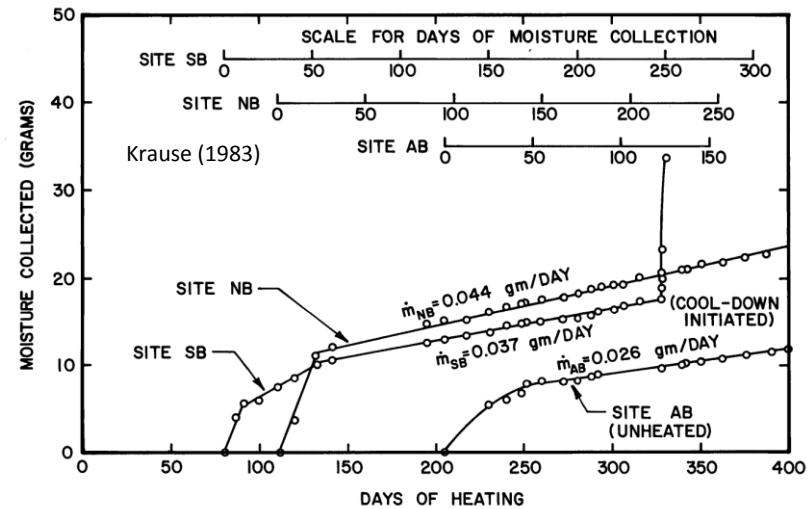
- $k \approx 10^5$  increase during cool down

### ■ Learned:

- Brine inclusion flow not significant (salt is porous medium)
- $k$  increase @ cooling allows brine flow



Krause & Gnirk (1982)



# Used Fuel Disposition

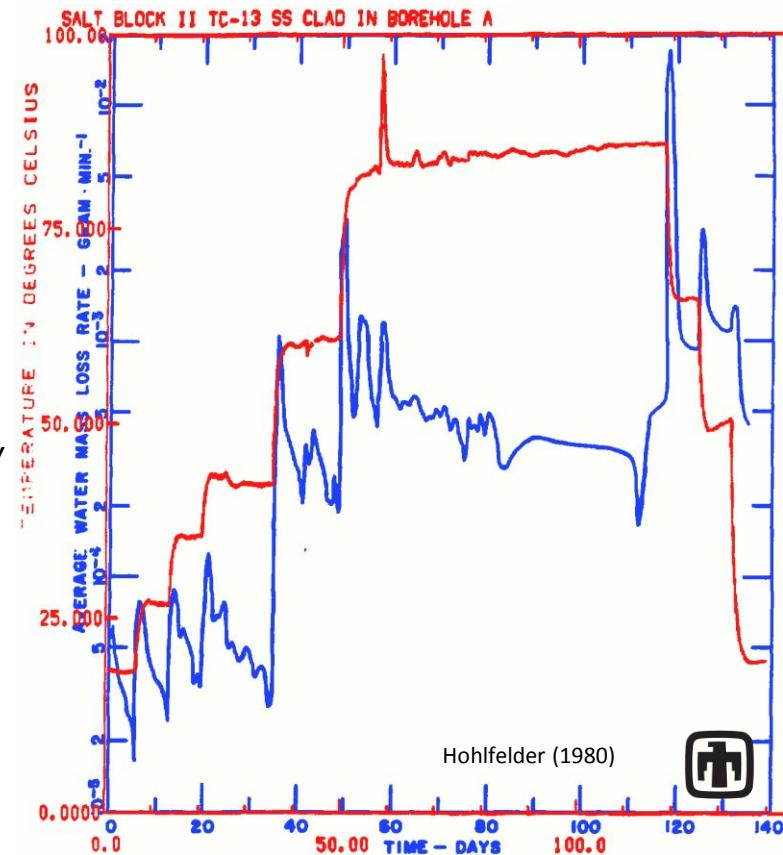
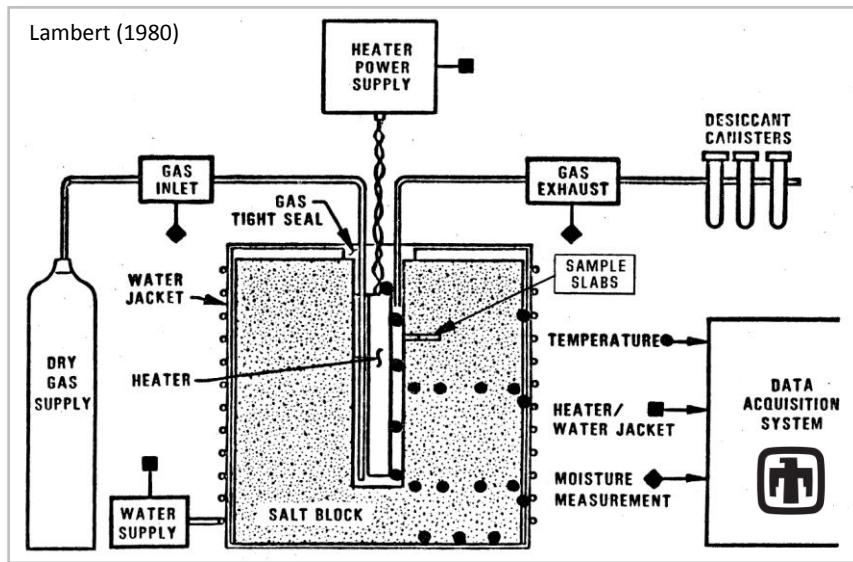
## SNL Lab Test: Salt Block II ('78 – '79)

- 1 m salt cylinder (1700 kg)
- Axially heated/cooled in steps
- High-frequency monitoring

- Brine inflow to heater borehole
- Temperature distribution

### Learned:

- Thermal response simple, brine flow requires new conceptual model



Temperature changes lead to spikes in brine inflow

Largest spike in brine inflow @ first cool-down step

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## SNL Lab Test: Salt Cracker ('80)

### ■ Two 1.6-kg salt cylinders

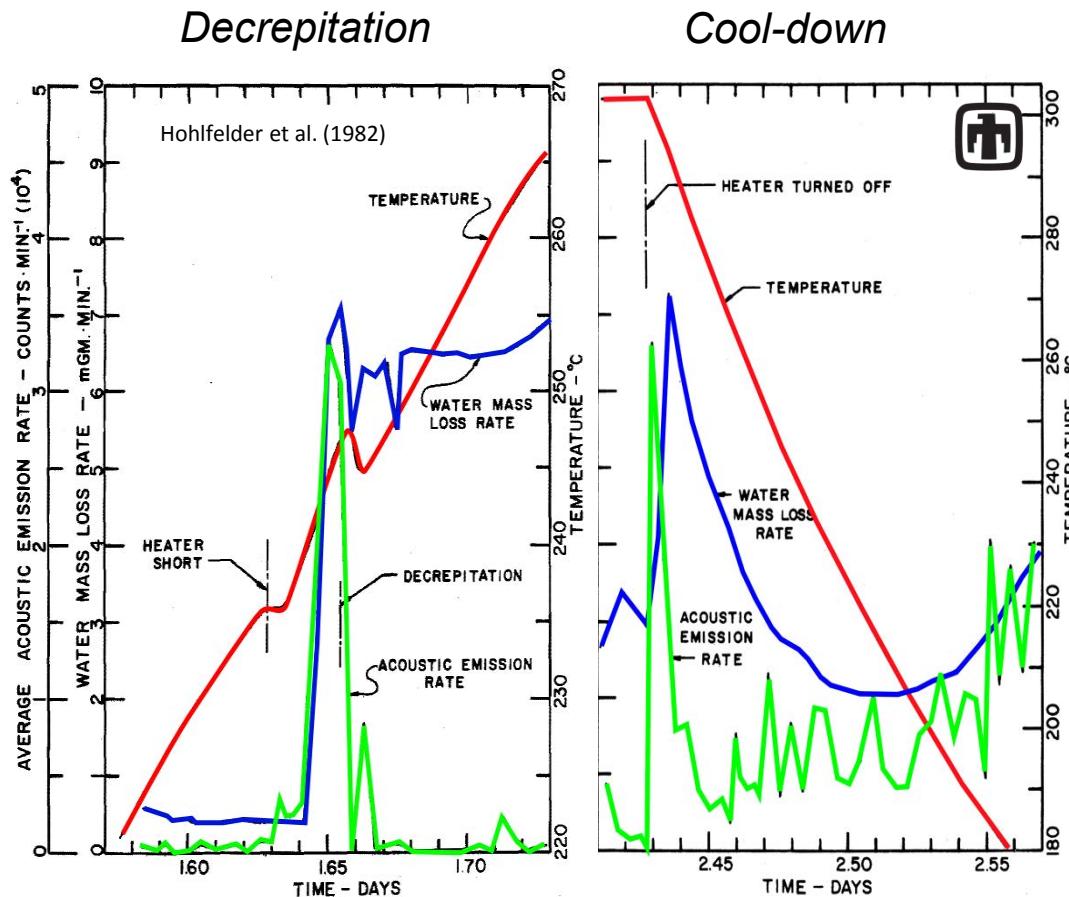
### ■ Heated to 200 & 300 °C

### ■ Brine Release Events

- Decrepitation (inclusions)
- Cool down (increase in  $k$  and porosity ( $n$ ))
- Increase in heater power (differential thermal expansion)

### ■ Learned:

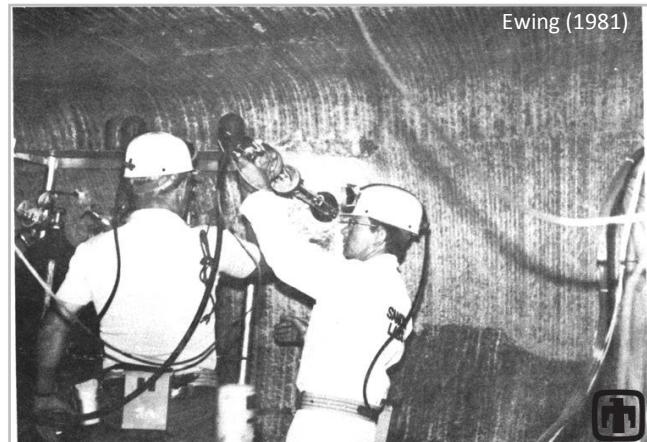
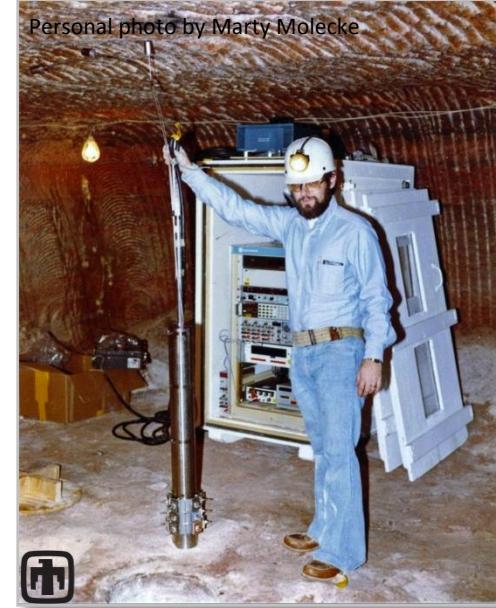
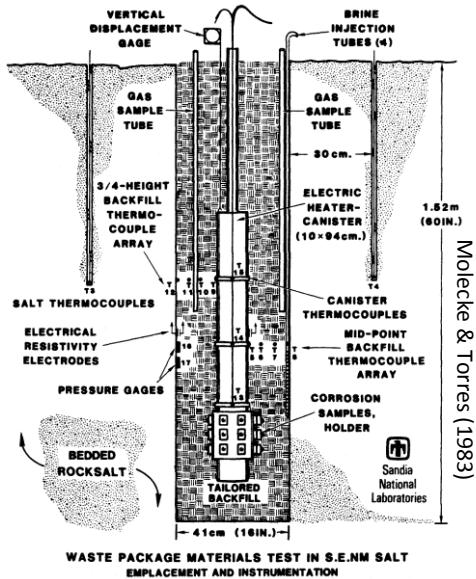
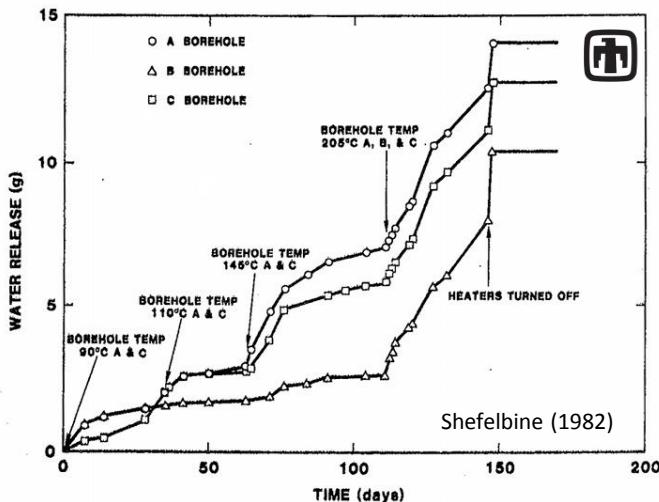
- Acoustic emissions reveal salt microfracturing
- Brine release @ cooling, even after decrepitation



# Used Fuel Disposition

## SNL Pre-WIPP Potash Mine Tests ('80 – '81)

- Tests conducted in Miss. Chem. Company Potash Mine before 1st WIPP shaft
- Waste package material testing
- Heater/brine inflow testing
- Instrumentation “dry run” for WIPP
- Learned:
  - Difficulties of working underground



# Used Fuel Disposition

# ***Waste Isolation Pilot Plant (WIPP)\****

## ■ *3 Primary SNL DHLW Test Programs*

- for future Deaf Smith site

## ■ *Thermal/Structural Interactions (TSI)*

- Rooms A1-A3 (18 W/m<sup>2</sup> DHLW mockup)
- Room B (DHLW overtest)
- Room H (Heated axisymmetric pillar)
- Room D (Isothermal Room B)

## ■ *Waste Package Performance (WPP)*

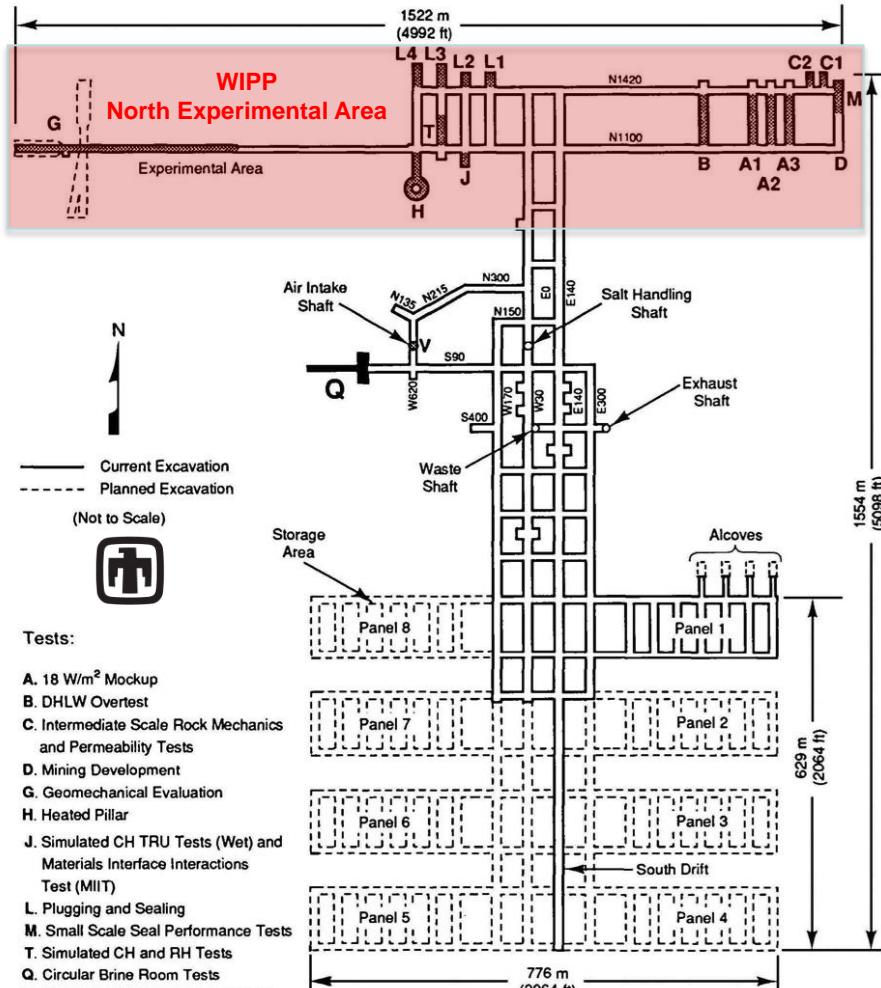
- DHLW materials tests in Rooms A1/B
  - *Waste Package materials tests*
  - *Borehole backfill materials tests*

## ■ *Plugging and Sealing Program (PSP)*

- Brine release in Rooms A1/B

## ■ *Many Non-DHLW programs*

- TRU tests Rooms J & T, brine flow in Q, etc.



\* Tyler et al. (1988) is comprehensive summary of DHLW testing at WIPP

Jensen et al. (1993)

# Used Fuel Disposition

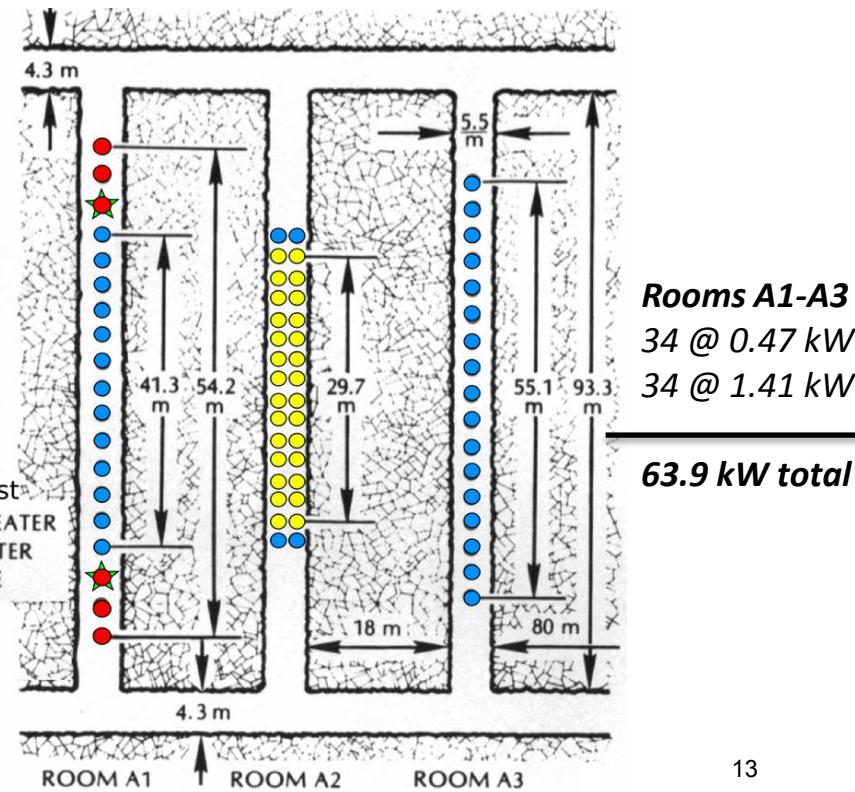
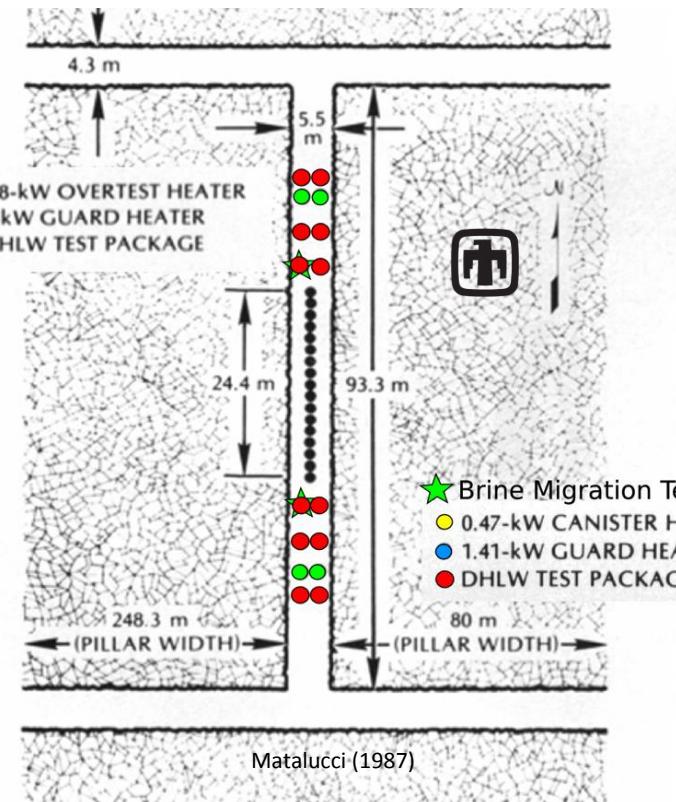
## SNL WIPP DHLW: Rooms A/B

- A Rooms: “design” DHLW thermal load (470 W heaters)
- Room B: “overtest” conditions (1800 W heaters)
- 4 brine migration boreholes
- 18 Waste Package Performance tests (7 retrieved)

**Room B**

17 @ 1.8 kW  
4 @ 4.0 kW  
8 @ 1.5 kW

**58.6 kW total**



# Used Fuel Disposition

## SNL WIPP DHLW: Rooms A/B/D

### ■ Rooms A/B:

- Temperature, differential creep, oriented stress (pressure), brine inflow, room closure, heat flux, and heater power.

### ■ Room D:

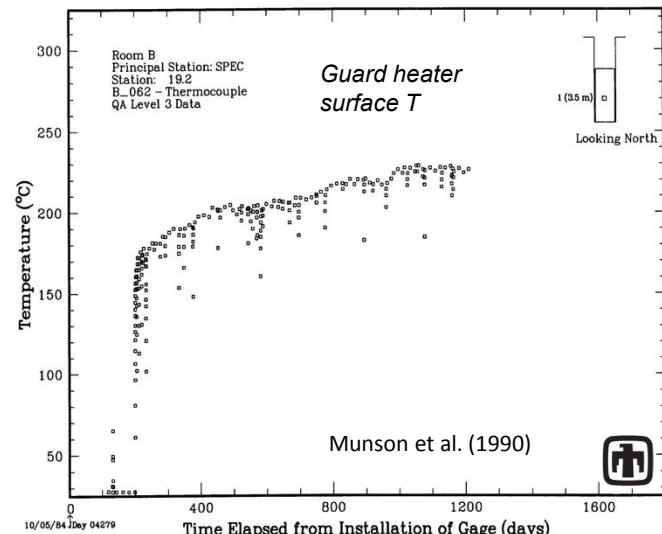
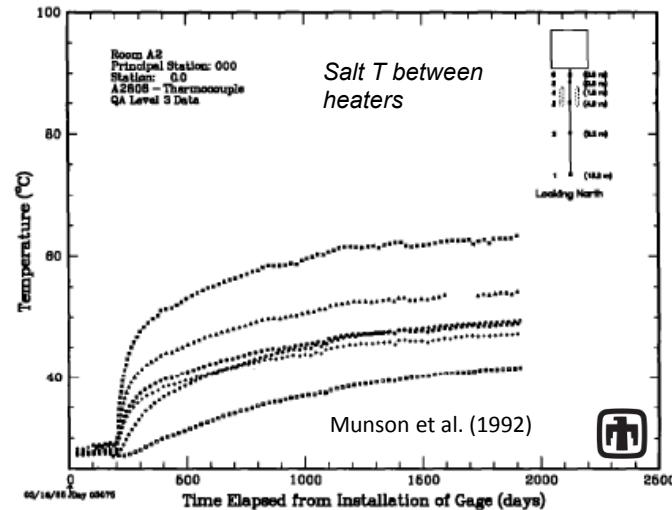
- Room B geometry w/ room closure obs.

### ■ Learned:

- *Roof failure in rooms preceded by rapid closure increase*
- *Ti alloy → corrosion-resistant canisters*

Room	Mining	Heat on	Heat off
D	Mar-Apr 1984		
B	May-June 1984	Apr 1985	Jan 1989
A2	June-July 1984	Oct 1985	Jun 1990
A1	Sep-Oct 1984	Oct 1985	July 1990
A3	Oct-Nov 1984	Oct 1985	Aug 1990

Kuhlman et al. (2012)



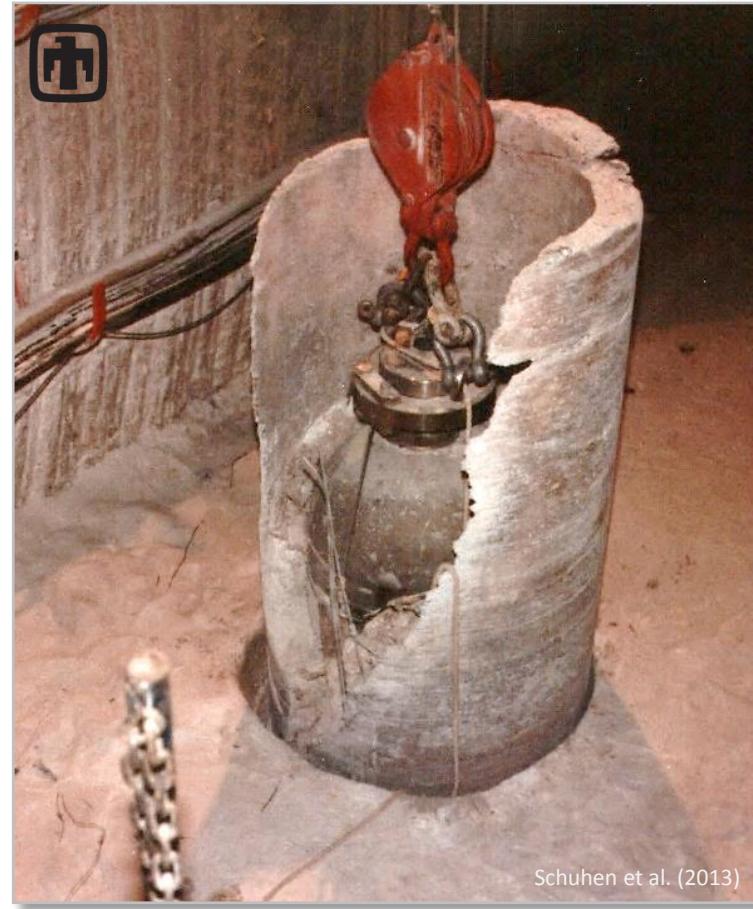
# Used Fuel Disposition

## *SNL WIPP DHLW: Room A2 ('85-'90)*



# Used Fuel Disposition

## *SNL WIPP DHLW: Room B ('85-'89)*



*Typical WPP DHLW canister in Room B at installation and removal  
Creep closure and salt crust deposition required overcoring to remove*

# Used Fuel Disposition

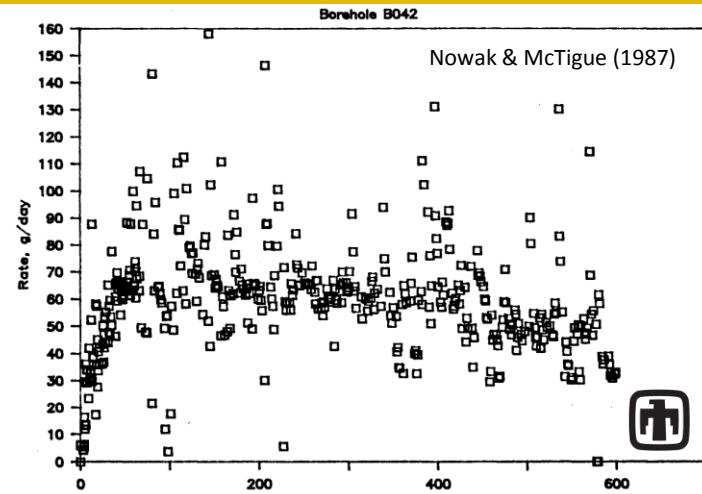
## SNL WIPP DHLW: A1/B Brine Release

### ■ Brine release:

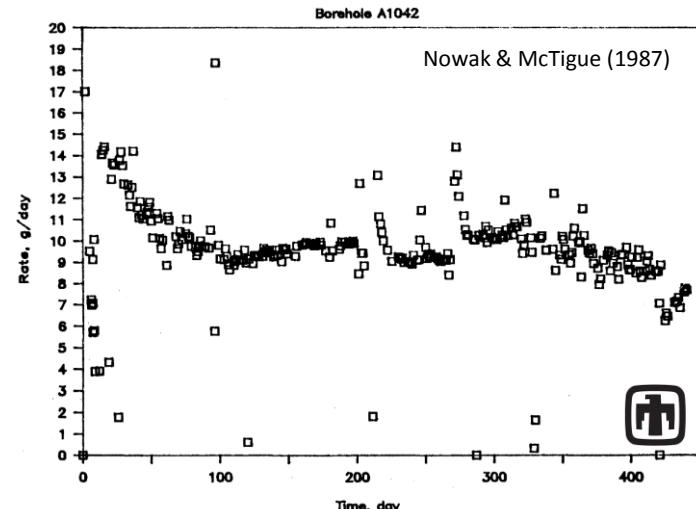
- Quantified before and during heating.
- Room B produced  $\approx 8\times$  more brine from same geology @  $\approx 3\times T$
- Significant brine inflow at Clay F

### ■ Learned:

- Vapor transport of brine in intact salt is insignificant
- Observed brine inflow consistent with salt rind observed @ heater retrieval.
- Thermo-poro-elasticitic model (McTigue, 1990) consistent with observed heated and isothermal flow (didn't consider brine inclusions)



Room B: 130° C, 35 L brine/borehole  
Room A1: 50° C, 4.5 L brine/borehole



- **Heated borehole consolidation of crushed salt ('87–'88)**
  - 5-boreholes: different grain-size distributions  
1.6 & 2.2-kW heaters
- **Borehole heater test (CPPS) ('91–'93)**
  - 4-kW heater in 7-m borehole
  - Reached 200° C max T at heater
  - 7 months of heating
- **Gas/brine permeability tests ('94)**
- **Learned:**
  - *No backfill complicated heat transfer, while crushed salt simplified it*
  - *Viscoplasticity model needed to explain brine flow under some conditions*



- Kazan & Ghoreychi (1997) and Ghoreychi et al., (1992) are english-language summaries of this work

■ ***Borehole heater tests 1–6 ('68 – '85)***

- Early tests to
  - *Determine in situ thermal properties of halite / crushed salt*
  - *Demonstrate heater, thermocouple, and brine collection systems*
  - *Demonstrate geophysical methods to interrogate heated salt (Kessels et al., 1986)*

■ ***Heated deep borehole closure ('79 – '82)***

- Closure data (calipers) inside borehole during heating (Doeven et al., 1983)

■ ***Heated Brine Migration test ('83 – '85)***

■ ***High Activity Waste (HAW) heater test ('88 – '94)***

■ ***Crushed Salt Reconsolidation***

- Heated drift backfilled with crushed salt: TSDE ('90 – '04)
- Heated vertical boreholes: DEBORA-1/2 ('97 – '98)

# Used Fuel Disposition

## Asse Brine Migration

### ■ **Heated 4 borehole sets ('83 – '85):**

- 2 with  $^{60}\text{Co}$  sources
- 2 sealed (vs. 1 atm)

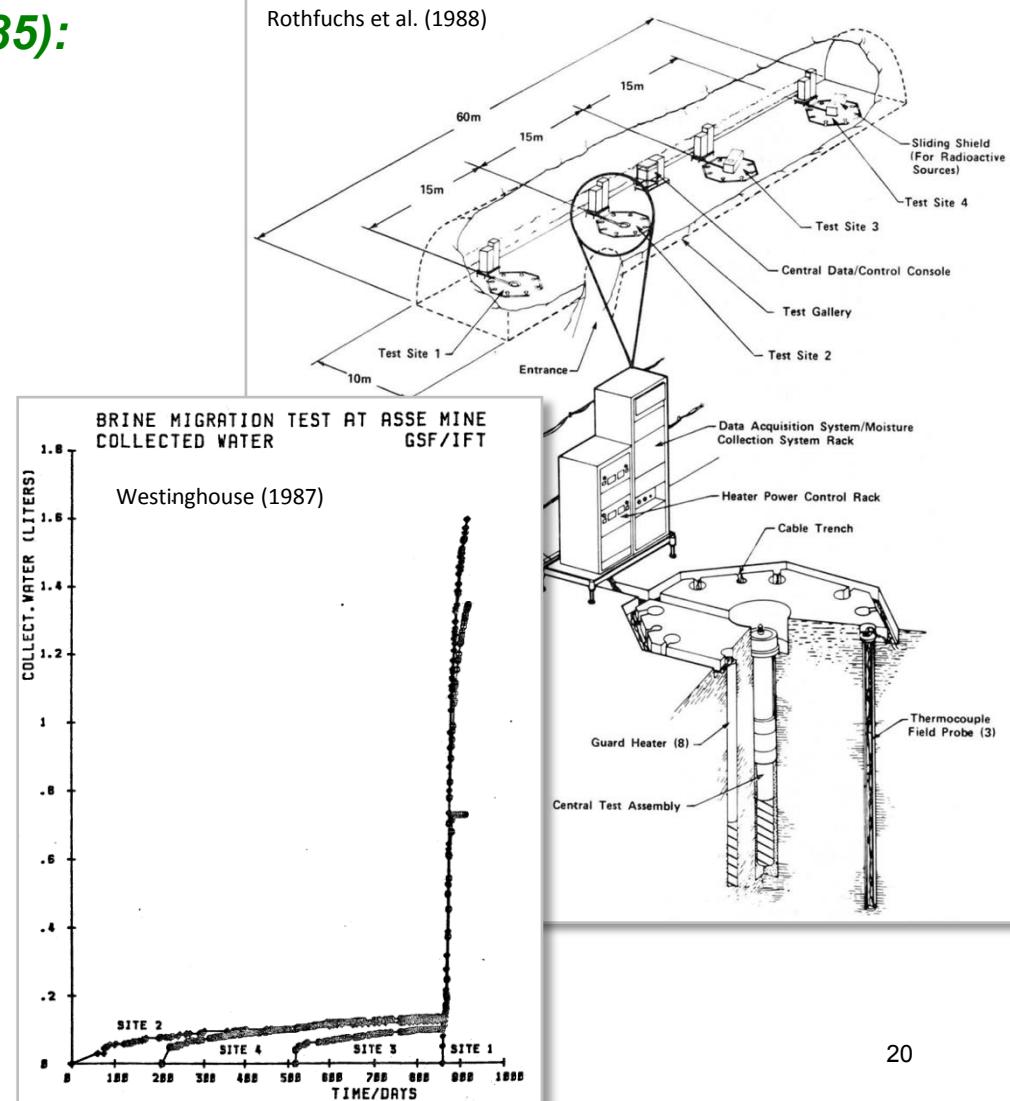
### ■ **Measured**

- Closure, Temperature
- Brine inflow
- Borehole gas content
- Acoustic emissions

### ■ **90% of brine collected during cooling**

### ■ **Learned:**

- Mechanical similar to bedded, brine flow  $\ll$  bedded
- Radiation had minimal effect

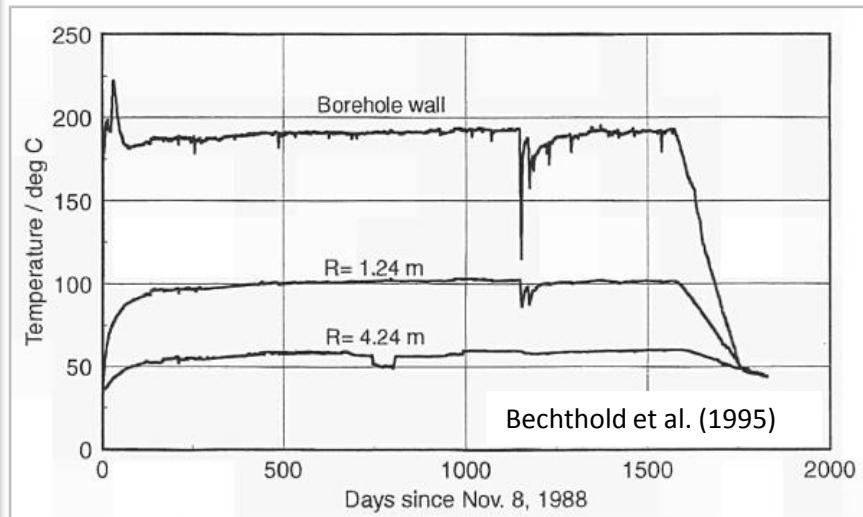
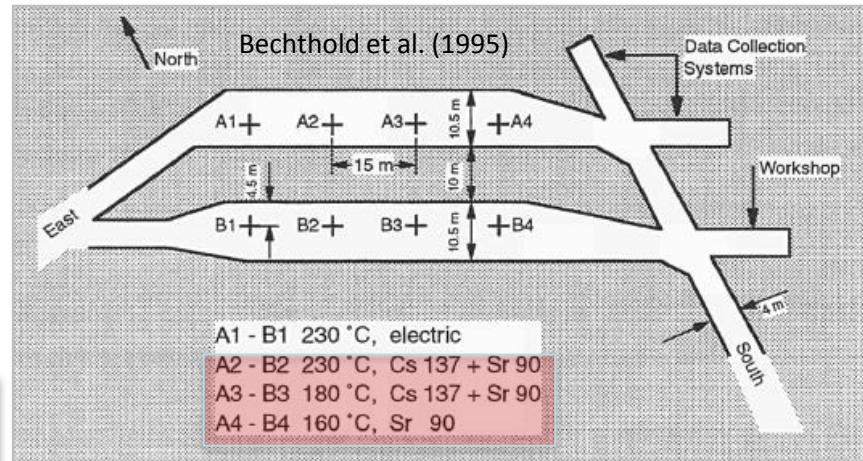
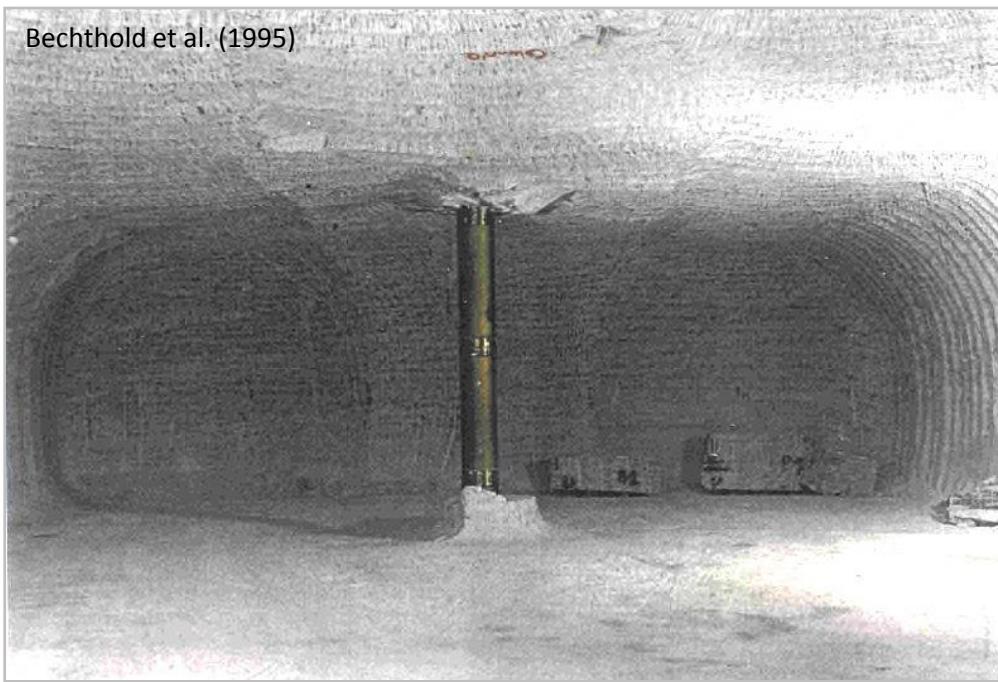


# Used Fuel Disposition

## Asse HAW Heater Tests

- **Abandoned 8-borehole High Activity Waste demonstration**
- **2 electrically heated boreholes**
  - A1 & B1 heated '88 – '92
- **Excavated B1 for corrosion study**

Bechthold et al. (1995)



# Used Fuel Disposition

## Asse Borehole Salt Reconsolidation

### Measured

- Corrosion,
- Temperature, pressure,
- Borehole convergence,
- Crushed salt  $k$  &  $n$

### DEBORA-1 ('97 – '98)

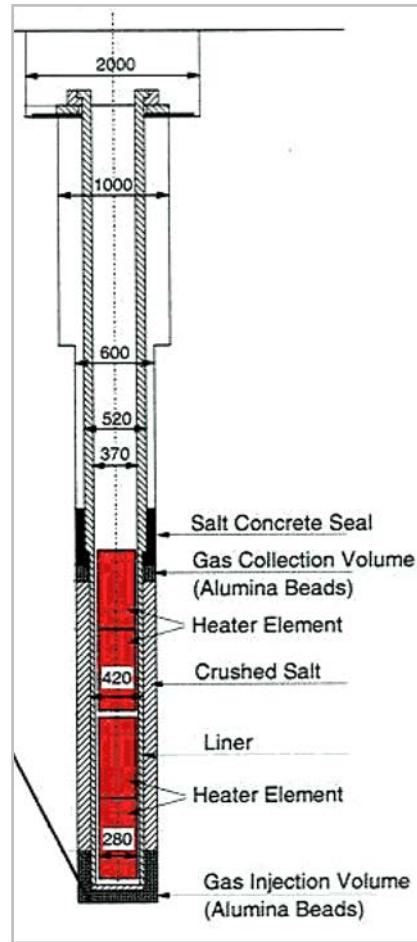
- 9-kW heater in 15-m borehole

### DEBORA-2 ('97 – '98)

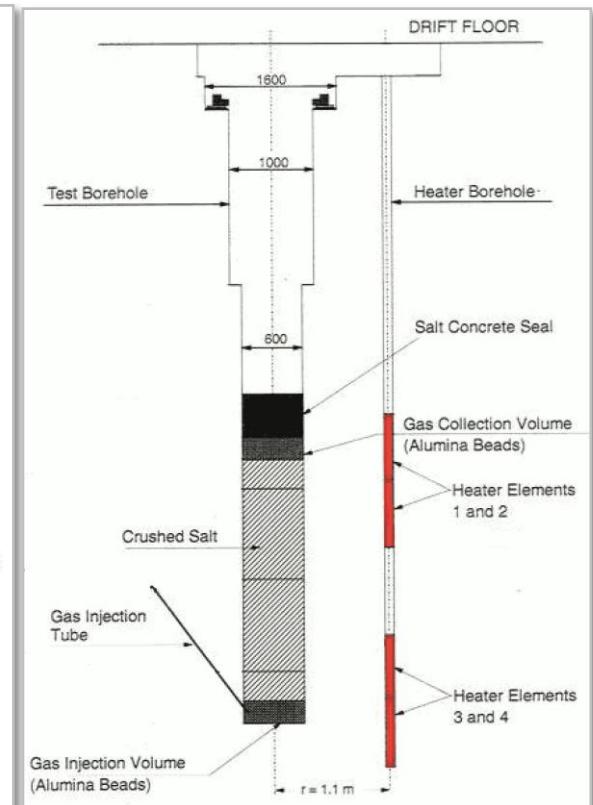
- 15-kW heaters around 15-m borehole

### Learned:

- Crushed salt reconsolidated significantly in months in boreholes



**DEBORA-1**  
 $n: 38\% \rightarrow 9\%$   
 $k: 5 \times 10^{-12} \rightarrow 7 \times 10^{-14} \text{ m}^2$



**DEBORA-2**  
 $n: 37\% \rightarrow 12\%$   
 $k: 1 \times 10^{-10} \rightarrow 4 \times 10^{-13} \text{ m}^2$

Bechthold et al. (1999)

## ■ Thermal Simulation of Drift Emplacement (TSDE)

### ■ Six Pollux casks

- $1.5\text{ m} \times 5\text{ m}$
- 6.4-kW heaters

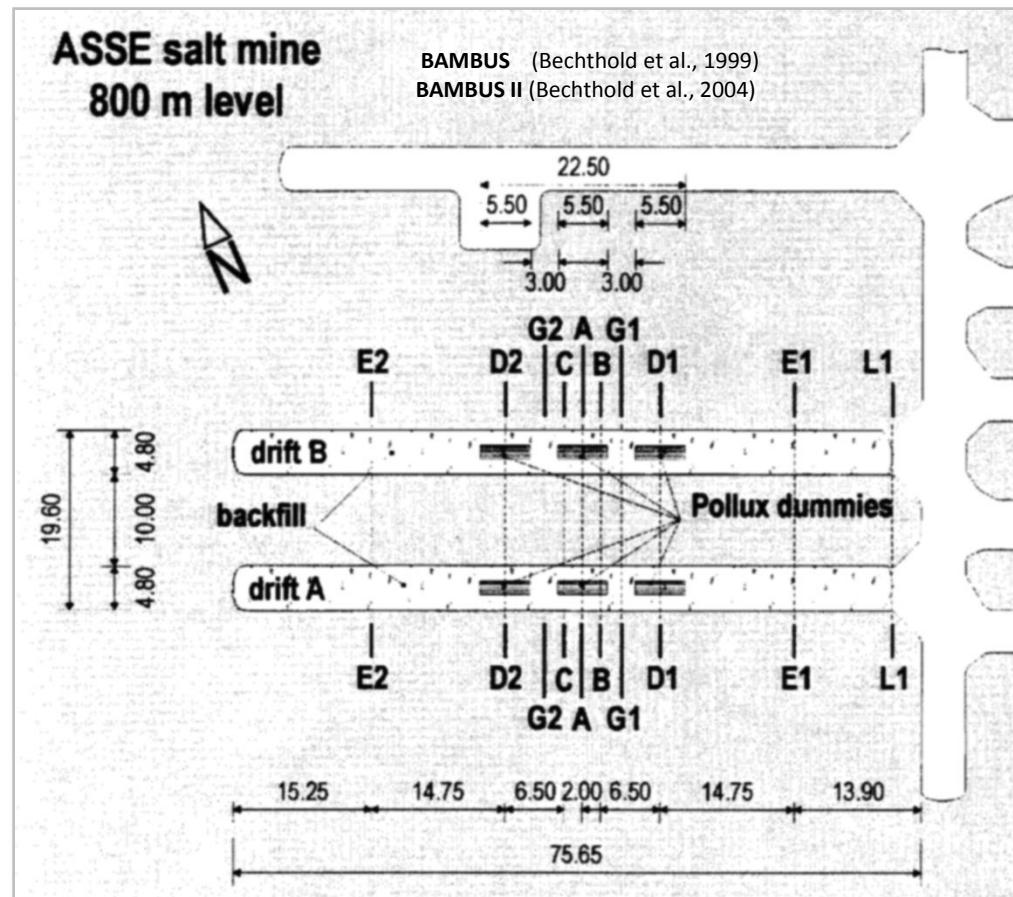
### ■ Backfilled to roof

### ■ Large thermal-mechanical time series collected

### ■ Post-test excavation data

### ■ Learned:

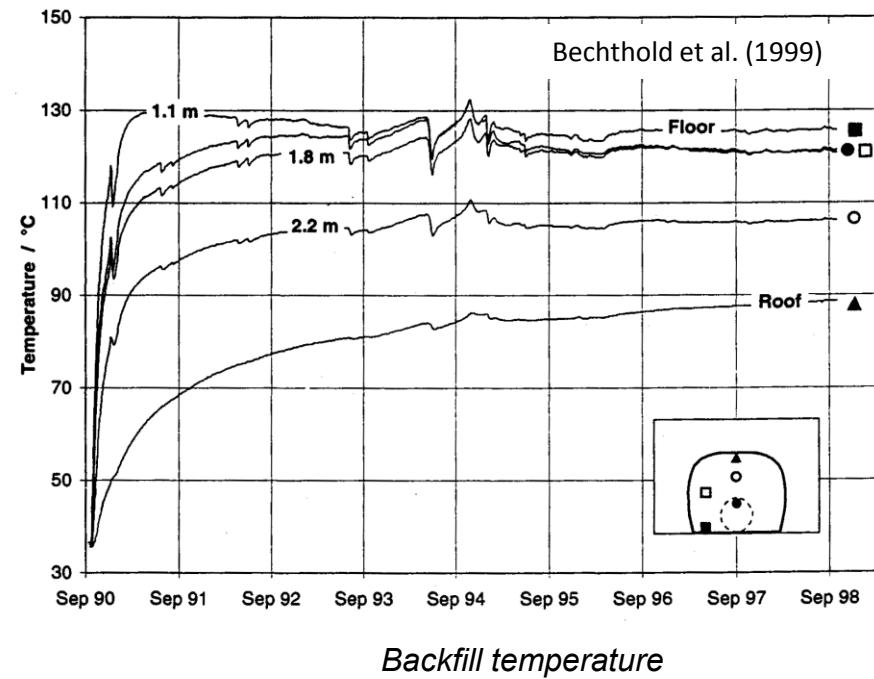
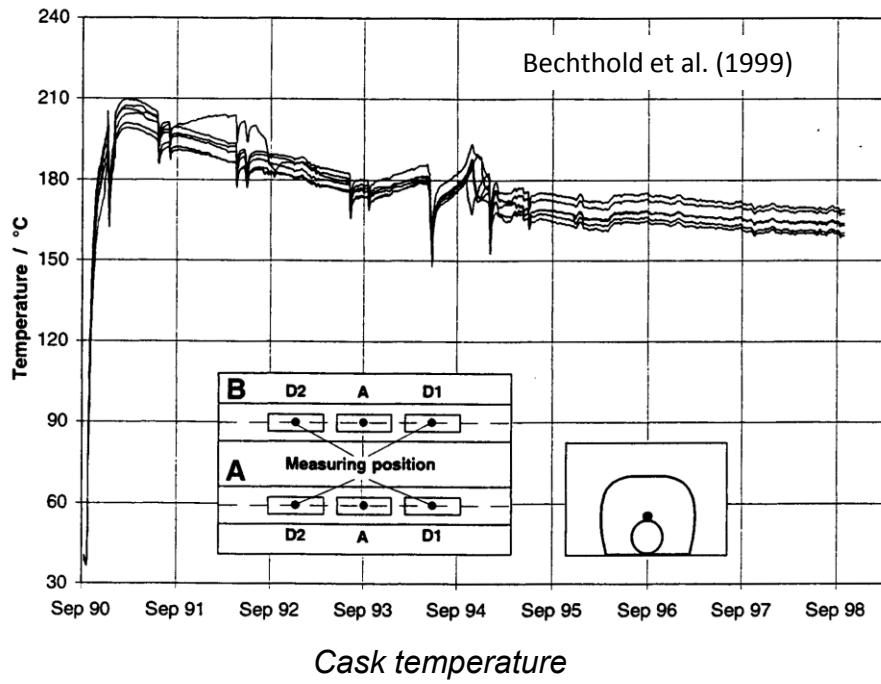
- Crushed salt reconsolidation less than in boreholes
- Extensive *in situ* validation dataset



■ **Heater  $T$  decreased (at constant power)**

- Backfill  $\alpha$  increasing with decreasing porosity
- Non-linear thermal conductivity:  $\alpha(T) \approx \alpha_0(T-T_0)^{-1.1}$

■ **Steady-state  $T$  reached near heaters (but not at roof)**



■ ***Salt Vault presented 1st safety case for HLW in salt (1971)***

- Culmination of 10+ years of laboratory and field testing
- Bradshaw & McClain (1971) summarized technical basis
- AEC proposed Lyons, KS as pilot-scale site for heat-generating waste

■ ***NRC Summary***

- Geoscience database for nuclear waste repositories (salt, granite, clay) (Isherwood, 1981)

■ ***Deaf Smith Site Characterization Plan (DOE, 1988; 10 vol.)***

- Regional site description
- Salt properties determined from core & other sites
- Site Investigation Plan (very detailed)

■ ***Gorleben safety case: ISIBEL project (2006—2010)***

- Site characterization
- Disposal system design
- Weber et al. (2011) summarized safety case.

■ ***Recent UFD/SNL historical testing and technical basis summaries***

- Kuhlman et al. (2012), Kuhlman & Malama (2013), Kuhlman & Sevougian (2013)

- **Technical basis for heat-generating waste in salt is not new**
- **Thermal-mechanical behavior is well known**
- **Modern numerical models**
  - Allow non-linear and coupled processes
  - Must be benchmarked against data
  - Not technical basis, but important tools
- **Long-term viability of salt repository:**
  - Salt deposit provides long-term containment
  - Shaft seals ensure containment uncompromised
    - *Seal emplacement*
    - *Reconsolidation of backfill*
  - Other repository features of secondary safety case importance
    - *Waste forms/waste packages*
    - *Brine migration into and through excavation*

Test Locale	<u>Bedded</u> vs. <u>Domal</u>	<u>Crushed</u> vs. <u>Intact</u>	<u>Borehole</u> vs. <u>In-Drift</u>
Salt Vault	<b>B</b>	I	<b>B</b>
Avery Island	<b>D</b>	I	<b>B</b>
WIPP DHLW	<b>B</b>	I	<b>B</b>
Amélie	<b>B</b>	<b>C + I</b>	<b>B</b>
Asse	<b>D</b>	<b>C + I</b>	<b>B + D</b>
?	<b>B</b>	<b>C</b>	<b>D</b>

# Used Fuel Disposition

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